




Impact of Brazed Joint Defects on Operability of ITER Divertor Dome Plasma Facing Units

A. Rybikov^{1,2} , A. Eremkin¹, A. Komarov¹, V. Kuznetsov¹, A. Volodin¹, P. Piskarev¹ ,
I. Bogdanov¹, V. Tanchuk¹, S. Grigoriev¹, M. Dorogov² , N. Arkhipov³

¹ JSC «NIIIEFA» (Efremov Institute), Doroga na Metallostroy, 3, Metallostroy, St. Petersburg, 196641, Russia

² Institute of Advanced Data Transfer Systems, ITMO University, Kronverkskiy pr., 49, lit. A, St. Petersburg, 197101, Russia

³ Institution «Project Center ITER», Raspletina St., 11, bldg. 2, Moscow, 123182, Russia

Received: September 09, 2025

Corresponding author: [A. Rybikov](mailto:A.Rybikov@niiiefa.ru)

Abstract. The study presents the results of numerical simulation and experiments of thermal processes in the mock-ups of the ITER divertor dome plasma facing units (PFU). The divertor is a critical component subjected to extreme thermal loads which imposes high requirements on the reliability of its PFUs. The purpose of the study is to assess the impact of the parameters (geometry, size, localization) of defects in the brazed joint of tungsten-copper armour and bronze heat sink under heat loads $q_s = 1\text{--}6 \text{ MW/m}^2$. The simulation by the finite-element method was performed for the PFU configurations with defects of various shapes and sizes. The results were compared with the experimental data of thermal tests. It was found that an increase in the defect size leads to an increase in the armour temperature: at the maximum defect area (50% of the joint area), the local temperature increases by 55%, and the average temperature by 40% ($q_s = 6 \text{ MW/m}^2$). The defect shape (rectangular/triangular) has a minor effect: deviations do not exceed 1.4% for the maximum and 10% for the average values. The longitudinal castellation (electrical discharge machining) of tiles in the model can affect the temperature distribution. The maximum armour surface temperature for all versions did not exceed 1200 °C. At loads $\leq 4 \text{ MW/m}^2$, cooling remains convective, while at the higher loads, local boiling zones occur without transition to a critical heat flux. The comparison between the calculated and experimental results has detected the similarity in the temperature distributions.

Acknowledgements. The materials of this paper are based on the work results under Contract No. 17706413348240000190/55-24/01 of 20.06.2024 for R&D “Development of technology and pilot production of components for the Dome and First Wall panels and performance of thermal

tests of divertor elements for the ITER reactor in 2024” concluded between the Institution “Project Center ITER” and JSC “NIIIEFA”.

Citation: Rev. Adv. Mater. Technol., 2025, vol. 7, no. 3, pp. 184–197

View online: <https://doi.org/10.17586/2687-0568-2025-7-3-184-197>

View Table of Contents: <https://reviewsamt.com/issues>

REFERENCES

1. N. Litunovsky, E. Alekseenko, A. Makhankov, I. Mazul, Development of the armoring technique for ITER Divertor Dome, *Fusion Engineering and Design*, 2011, vol. 86, no. 9–11, pp. 1749–1752.
2. A.A. Rybikov, N.A. Kuznetsov, N. Litunovsky, A. Makhankov, Development and testing of the technology for coating plasma-facing components of the ITER divertor central assembly, in: *Materials of III International Conference “Problems of Thermonuclear Energy and Plasma Technology”*, Izdatelskiy Dom MEI, Moscow, 2023, pp. 93–94 (in Russian).
3. V. Tanchuk, S. Grigoriev, A. Makhankov, K. Senik, N. Yablokov, M. Belenky, M. Blinov, M. Lebedev, B. Fokin, Experimental and numerical evaluation of IR thermography method for Final Acceptance Tests of the ITER divertor dome, *Fusion Engineering and Design*, 2014, vol. 89, no. 7–8, pp. 925–931.
4. R. Tivey, T. Ando, A. Antipenkov, V. Barabash, S. Chiochio, G. Federici, C. Ibbott, R. Jakeman, G. Janeschitz, R. Raffray, M. Akiba, I. Mazul, H. Pacher, M. Ulrickson, G. Vieider, ITER divertor, design issues and research and development, *Fusion Engineering and Design*, 1999, vol. 46, no. 2–4, pp. 207–220.
5. E. Visca, F. Escourbiac, S. Libera, A. Mancini, G. Mazzone, M. Merola, A. Pizzuto, Testing of high heat flux components manufactured by ENEA for ITER divertor, *Fusion Engineering and Design*, 2009, vol. 84, no. 2–6, pp. 309–313.
6. J. Schlosser, F. Escourbiac, M. Merola, S. Fouquet, P. Bayetti, J. Cordier, A. Grosman, M. Missirlan, R. Tivey, M. Rödiger, Technologies for ITER divertor vertical target plasma facing components, *Nuclear Fusion*, 2005, vol. 45, no. 6, pp. 512–518.
7. S.N. Mazaev, T. Gurieva, A. Lapin, A. Makhankov, V. Mirgorodsky, S. Natochev, O. Nomokonova, I. Vlasov, A. Ignatov, Laser welding of plasma facing units for ITER divertor Dome manufacturing, *2011 IEEE/NPSS 24th Symposium on Fusion Engineering*, Chicago, IL, USA, 2011, pp. 1–5.
8. T. Hirai, V. Barabash, F. Escourbiac, A. Fedosov, L. Ferrand, S. Gicquel, T. Jokinen, V. Komarov, A. Martin, M. Merola, Design and Integration of ITER Divertor Components, *Advances in Science and Technology*, 2010, vol. 73, pp. 1–10.
9. J. Davis, V. Barabash, L. Plöchl, K. Slattery, Assessment of tungsten for use in the ITER plasma facing components, *Journal of Nuclear Materials*, 1998, vol. 258–263, pp. 308–312.
10. T. Hirai, F. Escourbiac, V. Barabash, A. Durocher, A. Fedosov, L. Ferrand, T. Jokinen, V. Komarov, M. Merola, S. Carpentier-Chouchana, N. Arkhipov, V. Kuznetsov, A. Volodin, S. Suzuki, K. Ezato, Y. Seki, B. Riccardi, M. Bednarek, P. Gavila, Status of technology R&D for the ITER tungsten divertor monoblock, *Journal of Nuclear Materials*, 2015, vol. 463, pp. 1248–1251.
11. T. Hirai, K. Ezato, P. Majerus, ITER Relevant High Heat Flux Testing on Plasma Facing Surfaces, *Materials Transactions*, 2005, vol. 46, no. 3, pp. 412–424.
12. M. Missirlan, M. Richou, B. Riccardi, P. Gavila, T. Loarer, S. Constans, The heat removal capability of actively cooled plasma-facing components for the ITER divertor, *Physica Scripta*, 2011, vol. 2011, no. T145, art. no. 014080.

13. N. Litunovsky, E. Alekseenko, V. Kuznetsov, D. Lyanzberg, A. Makhankov, R. Rulev, Repair of manufacturing defects in the armor of plasma facing units of the ITER Divertor Dome, *Fusion Engineering and Design*, 2013, vol. 88, no. 9–10, pp. 1739–1743.
14. P. Piskarev, R. Rulev, I.V. Mazul, A.V. Krasilnikov, A.A. Pisarev, B. Kuteev, M.S. Kolesnik, V.V. Dushik, S. Bobrov, N.V. Montak, A.A. Rybikov, T.N. Bukatin, Coatings on a First Wall Plasma-Facing Surface: Analysis and High Heat Flux Testing on the Tsefey-M E-Beam Facility, *Physics of Atomic Nuclei*, 2024, vol. 87 (suppl. 1), pp. S118–S128.
15. A. Gervash, R. Giniyatulin, T. Guryeva, D. Glazunov, V. Kuznetsov, I. Mazul, P. Ogursky, P. Piskarev, V. Safronov, R. Eaton, R. Raffray, O.N. Sevryukov, The development of technology of Be/CuCrZr joining using induction brazing, *Fusion Engineering and Design*, 2019, vol. 146, pp. 2292–2296.
16. P.Yu. Piskarev, I.V. Mazul, L.E. Zakharov, G.M. Tarasyuk, M.S. Kolesnik, R.V. Rulev, A.Yu. Ogursky, A.A. Gervash, V.V. Ruzanov, Yu.M. Gasparyan, A.A. Pisarev, Fabrication and thermal tests of SS/Cu bimetal plate for use in the concept of flowing liquid lithium layer in tokamak limiters and divertors, *Fusion Engineering and Design*, 2022, vol. 184, art. no. 113313.
17. A.A. Rybikov, N.A. Kuznetsov, N.V. Litunovsky, A.N. Makhankov, P.Yu. Piskarev, A.V. Eremkin, I.P. Bogdanov, V.E. Kuznetsov, A.V. Volodin, N.I. Arkhipov, Determination of maximum permissible defect during brazing of tungsten-copper armour tiles for ITER divertor dome plasma-facing units, *Problems of Atomic Science and Technology, Series Thermonuclear Fusion*, 2025, vol. 2, in press.
18. N.V. Litunovsky, A.N. Makhankov, *Device for fixing and pressing parts during high-temperature vacuum brazing in resistance furnaces*, RU patent no. RU202604U1, 2021 (in Russian).
19. A. Eremkin, A. Volodin, A. Kokoulin, A. Komarov, V. Kuznetsov, A. Malyshev, N. Stepanov, Test facility for experimental investigation of ITER divertor components behavior under high heat fluxes, *Problems of Atomic Science and Technology, Series Thermonuclear Fusion*, 2020, vol. 43, no. 4, pp. 5–14 (in Russian).
20. H. Shim, J. Kim, M. Cheon, S. Pak, A comparison study on the derivation of in-structure FRS during seismic events for application of ITER upper port 18, *Fusion Engineering and Design*, 2023, vol. 191, art. no. 113771.
21. G. Aiello, M. Gagliardi, A. Meier, G. Saibene, T. Scherer, S. Schreck, P. Spaeh, D. Strauss, A. Vaccaro, ITER Torus Diamond Window Unit: FEM analyses and impact on the design, *IEEE Transactions on Plasma Science*, 2019, vol. 47, no. 7, pp. 3289–3297.
22. C. Vidal, R. Luis, B. Pereira, R.B. Ferreira, B. Gonçalves, S.B. Korsholm, A. Lopes, E.B. Klinkby, E. Nonbøl, M. Jessen, M. Salewski, J.J. Rasmussen, B. Lauritzen, A. Larsen, Thermo-structural analyses of the in-vessel components of the ITER collective Thomson scattering system, *Fusion Engineering and Design*, 2019, vol. 140, pp. 123–132.
23. D.M. Bachurina, P.V. Morokhov, R.S. Rasskazov, O.N. Sevryukov, B.A. Kalin, Effect of beryllium on the morphology of initial ingots and the structural phase state of fast-quenched STEMET 1108 brazing alloy ribbons, *Tsvetnye Metally*, 2022, no. 10, pp. 66–71 (in Russian).
24. B.A. Kalin, A.N. Suchkov, V. Fedotov, O.N. Sevryukov, A.A. Ivannikov, A.A. Gervash, Brazing of Be with CuCrZr-bronze using copper-based filler metal STEMET, *Nuclear Materials and Energy*, 2016, vol. 9, pp. 388–393.